

doc. Ing. Ondřej Klimo, Ph.D. FJFI CTU in Prague, ELI ERIC ondrej.klimo@fjfi.cvut.cz

Simulation techniques in hot plasma modeling 2023

Simulation techniques in hot plasma modeling

INTRODUCTION

□ What is Monte Carlo (MC) method?

- Experimental mathematics conclusions inferred from observations
- A last resort when doing <u>numerical integration</u> (useful in cases when other numerical techniques become prohibitively inefficient)
- □ A way to wastefully use CPU time very slow "stochastic" convergence $O(n^{1/2})$
- A method to search for solutions to mathematical problem using a statistical sampling with random numbers
 The MANIAC I at Los Alamos in 1952. Photo courtesy of LANL.

https://www.atomicheritage.org/history/computing-and-manhattan-project

Basic facts about MC particle transport method?

- MC method was developed by Stanislaw Ulam during the H-bomb project at Los Alamos Laboratory after Word War II.
- MC is often intuitive direct physical intuition used to get the algorithm
- MC has a sound mathematical basis



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INTEGRATION



https://en.wikipedia.org/wiki/Monte_Carlo_integration

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 $f_X(t) dt$

 $F_X(x) =$

BASIC CONCEPT

- □ Using a computer to generate random events:
 - □ <u>Uniformly distributed random numbers</u> U_[0,1]
 - General random numbers are obtained using U_[0,1] with various methods – e.g.
 - □ <u>inversion</u>
 - rejection
 - □ Need to generate random numbers X with any probability distribution function (PDF) - $f_X(x)$ or probability mass function (PMF) in the discrete case
 - A cumulative distribution function (CDF) (or cumulative mass function - CMF) is often used – inversion method



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APPLICATIONS

- Transport of energetic particles in particular in solid matter (examples)
- Electrons
 - \Box characteristic radiation e.g. K- α source
 - \Box Bremsstrahlung radiation γ -ray source
- - Isochoric heating (Warm Dense Matter)
 - Energy deposition in matter (hadron therapy)
 - Nuclear activation (nuclear reaction, ion diagnostics)
- Photons
 - Nuclear activation (nuclear reaction, photon diagnostics)
 - □ Attenuation (filtering/shielding)
 - □ Secondary e⁻ production (e.g. detector design)
 - Positron production Bethe-Heitler (e⁺ source)



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Be (2.5 mm)

1.0 cm

B=0.5

High

energy X rays

APPLICATIONS

 Example: Gamma detector design
 FLUKA simulation of production of Compton electrons

S. Singh et al., Rev. Sci. Instrum. 89, 085118 (2018), Compact high energy x-ray spectrometer based on forward Compton scattering for high intensity laser plasma experiments

- Transport of energetic particles in particular in solid matter (examples)
- Neutrons as secondary particles
 - Nuclear activation decay (diagnostics)
 - □ Fusion (neutron source)
- Sources, detectors and safety/filtering/shielding
- □ Other processes in particle simulations (e.g. PIC) Monte Carlo approach:
 - Binary Coulomb collisions
 - Ionization
 - \Box γ -radiation emission due to Bremsstrahlung or non-linear Compton scattering
 - □ e⁺e⁻ production



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DRUNK WALK

- □ The question is: How far will a drunk pedestrian be from the beginning (0, 0) after doing *N* steps of the same length but in random directions?
- At each step, the starting point is at the same time the end point of the previous step.
- □ The direction of the next step is given by the random angle (with respect to the previous direction given by angle θ_0)

$$\theta_W = 2\pi \cdot U_{[0,1]}$$

O – old angle in the lab frame
W – new angle in the walker frame
N – new angle in the lab frame

Transformation to the laboratory frame using the previous direction as

$$\begin{pmatrix} \cos \theta_O & -\sin \theta_O \\ \sin \theta_O & \cos \theta_O \end{pmatrix} \begin{pmatrix} \cos \theta_W \\ \sin \theta_W \end{pmatrix} = \begin{pmatrix} \cos \theta_N \\ \sin \theta_N \end{pmatrix}$$

The end point coordinates are thus

 $\begin{aligned} x_n &= x_{n-1} + step \cdot \cos \theta_N \\ y_n &= y_{n-1} + step \cdot \sin \theta_N \end{aligned}$

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DRUNK WALK

□ Intuitive - basic algorithm for particle transport:

- 1. Start at $(x_0 = 0, y_0 = 0), n = 0$
- 2. Sample initial direction θ_N
- 3. $n = n + 1, \theta_0 = \theta_N$
- 4. Sample new direction θ_W
- 5. Transform to the lab frame calculate θ_N
- 6. Calculate new coordinates (x_n, y_n)
- 7. Measure the distance from the origin $s = \sqrt{x_n^2 + y_n^2}$
- 8. Go to step 2. and repeat
- Simulate for N pedestrians to get mean and standard deviation (variance)



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PARTICLE TRANSPORT

□ What is different in particle transport:

- □ Step length is changing and has a given probability distribution
- □ 3D transport 2 scattering angles polar and azimuthal
- Particle loses energy
- □ Particle may cross the boundary between objects from with different properties
- Different events may occur at the end point (ionization, photon emission, nuclear excitation, absorption of the particle)
- Secondary particles may emerge
- Different quantities are measured during the transport

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PARTICLE TRANSPORT

- □ Intuitive basic algorithm for particle transport:
 - 1. Sample the source for a particle (E, Ω)
 - 2. Calculate the mean free path (MFP) λ_{mfp}
 - 3. Sample the exponential distribution to get new coordinates
 - 4. Sample the event at new coordinates
 - 5. Calculate energy loss and new direction of propagation $(E, \Omega) \rightarrow (E', \Omega')$
 - 6. Go to step 2. and repeat until particle absorbed or leaves the region of interest

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MATH BACKGROUND

□ Continuous path processes

- Diffusion process e.g. Brownian motion
- □ Transport equation e.g. Fokker-Planck equation

$$\frac{\partial}{\partial t}f(x,t) = -\frac{\partial}{\partial x}[D_1(x,t)f(x,t)] + \frac{\partial^2}{\partial x^2}[D_2(x,t)f(x,t)]$$

$$f(x,t) - \text{distribution function}$$

$$D_1 \text{ and } D_2 \text{ are the drift and diffusion coefficients.}$$

Appropriate e.g. for charged particle transport in plasmas – transport dominated by many small angle Coulomb collisions

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MATH BACKGROUND

□ Jump process – discrete path

- Can be described as Markov chain process (current state depends only on the previous one) and simulated using Monte Carlo method.
- Transport equation <u>Fredholm integral equation of the second kind</u>

$$\psi(\boldsymbol{p}) = S(\boldsymbol{p}) + \int \mathrm{d}\boldsymbol{p}' K(\boldsymbol{p}' \to \boldsymbol{p}) \,\psi(\boldsymbol{p}')$$

p - phase space coordinate $\psi(p)$ - probability density of finding the particle at the phase space coordinate S(p) - external source The integral accounts for transition of particles from other parts of the phase space.

 Appropriate e.g. for neutron transport, electron transport in solid target, fast ion transport – straight path between individual discrete collisions



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NEUTRON TRANSPORT

- Fredholm equation is a Boltzmann type transport equation
- □ Simplifying assumptions derivation for neutrons
 - □ neutrons point particles
 - □ neutrons neutral particles trajectory between interactions straight line
 - neutron-neutron interactions neglected
 - collisions are instantaneous
 - □ material properties are isotropic, known and time independent
 - expected or mean value of the neutron density distribution is considered
- **Quantities used to describe the transport**
 - □ <u>Neutron angular density</u> $n(r, E, \Omega, t)$ expected number of neutrons at position r with direction Ω and energy E at time t per unit volume per unit solid angle per unit energy
 - □ <u>Neutron angular flux</u> $\phi(\mathbf{r}, E, \mathbf{\Omega}, t)$ product $n(\mathbf{r}, E, \mathbf{\Omega}, t) \cdot v$ neutron angular density n and neutron velocity v

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NEUTRON TRANSPORT

Transport equation

 $\frac{\partial n}{\partial t} = \frac{1}{v} \frac{\partial \phi}{\partial t} = Production \ rate \ Q - Leakage \ rate \ L - Removal \ rate \ R$

- □ **Production rate** three source terms
 - \Box fission source Q_f
 - □ <u>independent external source</u> S

 \Box scattering source - Q_s - incident neutrons scatter from (E', Ω') to (E, Ω)

$$Q_{s} = \int_{0}^{\infty} dE' \int \Sigma_{s}(\boldsymbol{r}, E') C(\boldsymbol{r}, E' \to E, \boldsymbol{\Omega}' \to \boldsymbol{\Omega}) \cdot \boldsymbol{\phi}(\boldsymbol{r}, E', \boldsymbol{\Omega}', t) d\boldsymbol{\Omega}'$$

$$C - \text{probability of scattering from } (E', \boldsymbol{\Omega}') \text{ to } (E, \boldsymbol{\Omega})$$

$$\Sigma_{s} - \text{macroscopic scattering cross section}$$

□ Leakage rate

Y. Wu, Fusion Neutronics, Springer, 2017

 $L = \boldsymbol{\Omega} \cdot \nabla \phi$

Difference between the number of neutrons exiting the volume dV and the number of neutrons entering the volume dV per unit time

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NEUTRON TRANSPORT

Removal rate

- \Box neutrons absorbed in dV
- \Box neutrons scattered out from (*E*, Ω) in d*V*

$$\mathbf{R} = (\Sigma_s + \Sigma_a) \, \phi(\mathbf{r}, E, \mathbf{\Omega}, t) = \Sigma_t \, \phi(\mathbf{r}, E, \mathbf{\Omega}, t)$$

 Σ_s - macroscopic scattering cross section, Σ_a - macroscopic absorption cross section

In total

$$\frac{1}{v}\frac{\partial\phi}{\partial t} + \boldsymbol{\Omega}\cdot\nabla\phi + \Sigma_t\phi = S + \int_0^\infty \mathrm{d}E'\int\mathrm{d}\boldsymbol{\Omega}'\,\Sigma_s(\boldsymbol{r},E')C(\boldsymbol{r},E'\to E,\boldsymbol{\Omega}'\to\boldsymbol{\Omega})\cdot\phi(\boldsymbol{r},E',\boldsymbol{\Omega}',t)$$

□ Further simplification

□ Stationary solution - $\frac{\partial \phi}{\partial t} = 0$, no absorption - $\Sigma_a = 0$, $\Sigma_t = \Sigma_s$

 $\Box \ \psi = \Sigma_s \phi$ – particle collision density - average number of collisions

Integration along characteristics

$$\psi(\boldsymbol{r}, \boldsymbol{E}, \boldsymbol{\Omega}) = \int \mathrm{d}\boldsymbol{r}' \left[S + \int_0^\infty \mathrm{d}\boldsymbol{E}' \int \mathrm{d}\boldsymbol{\Omega}' \psi(\boldsymbol{r}', \boldsymbol{E}', \boldsymbol{\Omega}') C(\boldsymbol{r}', \boldsymbol{E}' \to \boldsymbol{E}, \boldsymbol{\Omega}' \to \boldsymbol{\Omega}) \right] T(\boldsymbol{r}' \to \boldsymbol{r}, \boldsymbol{E}, \boldsymbol{\Omega})$$

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MC SOLUTION

The equation to be solved by MC approach

□ Let
$$p = (r, E, \Omega)$$
 and $K(p' \rightarrow p) = C(r', E' \rightarrow E, \Omega' \rightarrow \Omega)T(r' \rightarrow r, E, \Omega)$

$$\psi(\boldsymbol{p}) = \int S(\boldsymbol{r}') T(\boldsymbol{r}' \to \boldsymbol{r}, \boldsymbol{E}, \boldsymbol{\Omega}) d\boldsymbol{r}' + \int \psi(\boldsymbol{p}') K(\boldsymbol{p}' \to \boldsymbol{p}) d\boldsymbol{p}'$$

- $\Box \psi$ particle collision density W. L. Dunn, J. K. Shultis, Exploring Monte Carlo Methods, Academic Press 2012
- \Box *S* source term
- □ C collision kernel
- □ T transport kernel
- Summary of assumptions
 - Static (time independent) homogeneous medium
 - Markovian
 - Particles transport independent of each other
 - Straight trajectories between collisions (no long-range forces)
- Superposition principle applicable

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MC SOLUTION \Box Expansion of ψ into components after 0,1,2, ..., k collisions

$$\psi(\boldsymbol{p}) = \sum_{k=0}^{\infty} \psi_k(\boldsymbol{p}), \text{ with } \psi_0(\boldsymbol{p}) = \int S(\boldsymbol{r}') T(\boldsymbol{r}' \to \boldsymbol{r}, \boldsymbol{E}, \boldsymbol{\Omega}) d\boldsymbol{r}'$$

As the process is Markovian

$$\psi_k(\boldsymbol{p}) = \int \psi_{k-1}(\boldsymbol{p}') K(\boldsymbol{p}' \to \boldsymbol{p}) \mathrm{d}\boldsymbol{p}'$$

- $\Box \psi_{k-1}(p')$ probability density of (k-1) collision at p'
- □ $K(p' \rightarrow p)$ conditional probability of (k) collision at p provided that the (k 1) collision was at p'

Monte Carlo solution

- 1. Randomly sample p' from $\psi_{k-1}(p')$
- 2. Randomly sample p from $K(p' \rightarrow p)$
- 3. If $p \in (p_i \mathrm{d}p_i, p_i + \mathrm{d}p_i)$ then $\psi_k(p_i) = \psi_k(p_i)$ +1
- 4. Repeat steps 1,2,3 *N* times
- 5. Monte Carlo solution is $\psi_k(p_i) = \psi_k(p_i)/N$

P. Vaz, Neutron transport simulation, Radiation Physics and Chemistry 78, 829-842 (2009).

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MC SOLUTION

□ A better approach – **Histories**

$$\psi_k(\boldsymbol{p}) = \int \psi_{k-1}(\boldsymbol{p}') K(\boldsymbol{p}' \to \boldsymbol{p}) d\boldsymbol{p}'$$
$$= \int \dots \int \psi_0(\boldsymbol{p}_0) K(\boldsymbol{p}_0 \to \boldsymbol{p}_1) \dots K(\boldsymbol{p}_{k-1} \to \boldsymbol{p}) d\boldsymbol{p}_0 \dots d\boldsymbol{p}_{k-1}$$

History - a sequence of states going from the source up to the "absorption" state Sample histories – 4 fast electrons propagating in Al target



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MC SOLUTION

- The histories are generated as follows
 - \Box Randomly sample source with PDF $\psi_0(p_0)$
 - □ Randomly sample the k^{th} transition with PDF $K(\mathbf{p}' \rightarrow \mathbf{p})$
- □ Having *M* histories phase space density measurements are possible

$$A = \int A(\boldsymbol{p}) \psi(\boldsymbol{p}) d\boldsymbol{p} = \frac{1}{M} \sum_{m=1}^{M} \left(\sum_{k=1}^{\infty} A(\boldsymbol{p}_{k,m}) \right)$$

- Measurable quantities are for example:
 - linear energy transfer (similar to dose) amount of energy ionizing particle transfers to the material per unit distance (action of radiation into matter)
 - ionizing events characteristic radiation emission
 - hard collisions bremsstrahlung emission, etc.

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ELECTRON SCATTERING

- □ Individual scattering events modelled single scattering model
- Scattering events
 - Elastic no energy loss scattering angle can be large different analytical formulas exist for electrons in the keV screened Rurherford cross section cross sections for a wider range of energies (Mott and Massey, The Theory of Atomic Collisions 1965)
 - □ Inelastic energy loss due to excitation or ionization (events with large scattering angle are unlikely as $\theta \sim \Delta E/E$ relative energy loss in collision) for cross sections see e.g. manual of the PENELOPE code
- □ Inelastic scattering not accounted for (not interested in ionization) energy loss calculated differently
- □ Screened Rutherford elastic scattering cross section

$$\sigma_R = 5210 \ \frac{Z^2}{E^2} \frac{4\pi}{\alpha(1+\alpha)} \left(\frac{E+511}{E+1024}\right)^2 \ \left[\frac{barn}{atom}\right]$$

electron kinetic energy E is in keV, Z – atomic number and α – screening parameter

D.C. Joy, Monte Carlo Modeling for Electron Microscopy and Microanalysis, Oxford, 1995

□ Total cross section integrated over all scattering angles, can be used to calculate the mean free path

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MEAN FREE PATH

- Mean free path average distance travelled by a moving particle between successive collisions, which modify its properties (E, Ω)
 - □ The exponential law:
 - \Box P(x) probability of not having an interaction after a distance x
 - $\Box w dx$ probability to having an interaction between x and x + dx
 - $\Box \ w = \sigma \times N$
 - \Box N number of target particles per unit volume
 - \Box σ microscopic interaction cross section



- Poisson process exponential distribution describes the time (distance) between events in a Poisson point process in probability theory and statistics
- □ Solution exponential distribution. $P(x) = \exp(-w \cdot x)$, P(0) = 1
- **D** Mean free path between interactions $\lambda_{mfp} = 1/w$

Ref: W.R. Leo, "Techniques for Nuclear and Particle Physics Experiments"

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MEAN FREE PATH

Mean free path between interactions
$$\lambda_{mfp} = \frac{1}{w} = \frac{1}{\sigma N}$$

- □ E.g. in solids, λ_{mfp} is of the order of tens nm for 100 keV electron and 10 times smaller for 10 keV electron
- How to sample MFP using cumulative distribution function (CDF) and inversion method CDF is monotonic function increasing from 0 to 1

$$F(x) = \frac{\int_0^d P(x) dx}{\int_0^\infty P(x) dx} = \frac{\int_0^d \exp(-x/\lambda_{mfp}) dx}{\int_0^\infty \exp(-x/\lambda_{mfp}) dx} = 1 - \exp\left(-\frac{s}{\lambda_{mfp}}\right)$$

Sampling of the distance traveled s

$$s = -\lambda_{mfp} \ln(1 - F(x)) = -\lambda_{mfp} \ln(U_{[0,1]}).$$
 If $F(x)$ is $U_{[0,1]}$ then $(1 - F(x))$ is too.

Different interactions

□ Different cross sections – different mean free path e.g. $\lambda_A \sim 1/\sigma_A$, $\lambda_B \sim 1/\sigma_B$

$$\Box \text{ Total mean free path - } \sigma_T = \sigma_A + \sigma_B \implies \frac{1}{\lambda_T} = \frac{1}{\lambda_A} + \frac{1}{\lambda_B}$$

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SCATTERING ANGLE

□ Scattering angle is given by the cross section differential in scattering angle

$$\sigma_{R}' = \frac{\mathrm{d}\sigma_{R}}{\mathrm{d}\Omega} = 5210 \frac{Z^{2}}{E^{2}} \left(\frac{E+511}{E+1024}\right)^{2} \frac{1}{(\sin^{2}(\vartheta/2)+\alpha)^{2}}$$
$$\sigma_{R} = \int_{0}^{2\pi} \mathrm{d}\varphi \int_{0}^{\pi} \mathrm{d}\vartheta \sin\vartheta \,\sigma_{R}'$$

Polar scattering angle is obtained as $U_{[0,1]} = \int_{0}^{2\pi} d\varphi \int_{0}^{\theta} d\vartheta \sin \vartheta \frac{\sigma_{R'}}{\sigma_{R}}$ Using the inversion method $\cos \theta = 1 - \frac{2\alpha R}{(1 + \alpha - R)}, R = U_{[0,1]}$

□ Azimuthal scattering angle is random, rotational symmetry of the collision frame

$$\varphi = 2\pi \cdot U_{[0,1]}$$

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ENERGY LOSS

Energy loss

- \Box Inelastic collisions dominates for lower *E* and lower *Z*
- Bremsstrahlung emission emission of photon during elastic collision
- Simulation of all energy loss events very complicated and time consuming
- □ Energy loss in individual collisions usually low so called soft events
- □ Energy loss averaged per unit path so called stopping power
- For solid targets and electrons Bethe stopping power
- □ For electron in keV range

$$\frac{\mathrm{d}E}{\mathrm{d}S} = -78500 \cdot \frac{Z}{AE} \cdot \ln\left(\frac{1.166E}{J}\right), \qquad S = s \cdot \rho$$

where s is the distance traveled in cm, ρ is the density in g/cm³, A is the atomic weight a both E and the mean ionization potential J are in keV (only for $E \gg J$)

□ Stopping powers and range tables can be found in Estar database of NIST, not now

NOTICE: Due to a lapse in government funding, this and almost all NIST-affiliated websites will be unavailable until further notice. Learn more NIST websites for programs using non-appropriated funds (<u>NVLAP</u> and <u>PSCR</u>) or those that are excepted from the shutdown (such as <u>NVD</u>) will continue to be available and updated. https://pml.nist.gov/PhysRefData/Star/Text/ESTAR.html

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SINGLE SCATTERING

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CONDENSED HISTORY

- With stopping power Continuous Slowing Down Approximation (CSDA) step size related only to events, where electron is deflected
- Simulating all elastic scattering events (single scattering) very inefficient many small angle deflections and only few large angle ones
- Condensed history technique (plural or multiple scattering) deflection events grouped together
- □ Multiple scattering step size not related λ_{MFP}
- □ Popular choice constant fractional energy loss per step, i.e. $\frac{\Delta E}{E}$ = const. e.g. 4%
- Condensed history Class II scheme
 - \Box Bremsstrahlung photon creation directly above an energy threshold E_{γ}
 - \Box Knock-on electrons above energy threshold E_{δ} treated by creation and transport
 - Sub-threshold processes accounted for in CSDA model and multiple scattering
- Depending on the application, condensed history "outperforms" single-scattering by a factor 10³ – 10⁵
- Radiation transport random process, energy loss fluctuations called "energy straggling" may be important

e.g. O. N. Vassiliev, Monte Carlo Methods for Radiation Transport Fundamentals and Advanced Topics, Springer, 2017

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VARIANCE REDUCTION

- The classical (so called) analog MC method works well if the probability of the situation of interest occurring is not very low
- Otherwise too time consuming and bad statistics
- Variance reduction techniques help to make it more efficient by
 - PDF biasing
 - Particle splitting
- Biasing PDF of unlikely events of interest is increased. Each particle is assigned with a statistical weight w to obtain unbiased results

$$w_{biased} = w_{unbiased} \frac{PDF_{unbiased}}{PDF_{biased}}$$

- □ If *PDF*_{biased} is increased, *w*_{biased} is decreased proportionally so the averaged outcome of the unbiased PDF is preserved. Examples are:
- □ Implicit capture (survival)
 - \Box Instead of "absorption" the particle is forced to continue and w_{biased} is reduced
 - \Box If w_{biased} too low Russian roulette either termination or increase of weight

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VARIANCE REDUCTION

- Forced collision in a small volume particle is split into 2
 - □ First one passes through, $w_1 = \exp(-s/\lambda_{mfp})$
 - □ Second one is forced to collide, $w_2 = (1 w_1)$
- D Particle splitting different particles contribute differently to the objective
- Increase the survivability of the "important" particles and eliminate "less important" ones. For example:
- <u>Geometric splitting with Russian roulette</u> volume partitioned into regions with different importance
- Particle moves:
 - □ From region of lower importance to higher importance splitting
 - From region of higher importance to lower importance Russian roulette decides whether the particle is killed

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e⁺/e⁻ INTERACTIONS

Possible interactions of electrons with matter in the range of interest

Elastic scattering

- Initial and final quantum states of target atom same
- Responsible for angular deflections
- Target recoil neglected
- Inelastic scattering
 - Produce electronic excitations and ionizations in the medium
 - Dominant energy loss mechanism for lower energies
 - Relaxation to ground state by emitting X-rays and Auger electrons
- Bremsstrahlung emission
 - Result of acceleration by the electrostatic field of atoms
 - Angular deflection accounted by elastic collision
 - \Box Photon energy in the range 0 to *E*
- Positron annihilation
 - Annihilation with the electrons in the medium by emission of two photons

F. Salvat, J. Fernández-Vera, J. Sempau,

PENELOPE-A code system for Monte Carlo simulation of electron and photon transport, OECD/NEA Data Bank. (2006).















Positron annihilation

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PHOTON INTERACTIONS

Dominant interactions in the energy range 0.1 keV – 1 GeV

Rayleigh (coherent) scattering

- □ By bound electrons without excitation elastic scattering
- Cross section related to Thomson scattering by free electron and atomic form factor
- Photoelectric effect
 - Photon absorbed by target atom, transition to excited state
 - □ Photon energy > ionization energy photoionization
 - Relaxation to ground state by emitting X-rays and Auger electrons

Compton scattering

- Photon absorbed by atomic electron and re-emited
- Active target electron ejected with finite kinetic energy
- Pair production
 - Absorption of photon in vicinity of a nucleus Bethe Heitler
 - □ Threshold process $2 m_e c^2$

















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PROTON INTERACTIONS

Important interactions of protons with matter in the range of interest

Elastic scattering

- Pure electromagnetic point-like nucleus and proton
- □ Residual includes strong interaction due to nucleus size effect
- Recoil important
- Inelastic scattering
 - Produce electronic excitations and ionizations in the medium
 - Dominant energy loss mechanism
 - Relaxation to ground state by emitting X-rays and Auger electrons

Nuclear reaction

- Less frequent but have more profound effect.
- Projectile proton enters the nucleus, which emits a proton, deuteron, triton, heavier ion or one or more neutrons

Bremsstrahlung emission

- Small effect on stopping of protons radiated power small
- Dosimetry simulations bremsstrahlung often neglected











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NEUTRON INTERACTIONS

- Neutrons are born fast, they slow down due to scattering (moderation) until they reach thermal energies and are absorbed by a target nucleus.
- Neutron energies
 - □ Fast (>100 keV)
 - □ Intermediate (10 eV 100 keV)
 - □ Slow (< 10 eV)
 - □ Thermal (0.025 eV)
- Interactions with atomic nuclei
 - Elastic Scattering most likely interaction, nuclear recoil
 - Inelastic Scattering excitation of the nucleus (emission of gamma rays), higher Z and high energy neutrons
 - □ Capture/absorption by the nucleus nuclear reaction
 - Unstable nucleus created deexcitation by emission of p, α, nucleus fragments





Neutron

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MC CODES

EGSnrc <u>https://nrc-cnrc.github.io/EGSnrc/</u>

- Particle interactions photons, electrons and positrons with kinetic energies between 1 keV and 10 GeV
- Software toolkit set of functions and subroutines for the simulation of coupled electron/photon transport
- Applications based on EGSnrc
 - BEAMnrc used to model medical linear accelerators
 - DOSXYZnrc dose distributions including geometries defined via CT images, incorporated into a number of clinical treatment planning codes

GEANT4 http://geant4.web.cern.ch

- Particle interactions electron, ion, muon, gamma ray, electromagnetic (EM), hadronic, and optical photons (many kinds of particles) very wide energy range
- Toolkit for simulation of passage of particles through matter with applications in high energy, nuclear and accelerator physics, medical and space science
 - MULASSES used to analyze radiation shielding for space missions
 - OpenGATE used for tomography applications

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MC CODES

MCNP <u>https://mcnp.lanl.gov</u>

- Particle interactions neutrons up to 20 MeV for all isotopes and up to 150 MeV for some, photons from 1 keV to 100 GeV and electrons from 1 keV to 1 GeV.
- Applications include radiation protection and dosimetry, shielding, radiography, medical physics, nuclear criticality safety, detector design and analysis, accelerator target design, fission and fusion reactor design, decontamination and decommissioning.
- PENELOPE <u>https://www.oecd-nea.org/tools/abstract/detail/nea-1525</u>
 - Particle interactions electron/positron-photon transport in energy range between 50 eV and 1 GeV.
 - Applications include electron backscattering, electron probe microanalysis and Xray generators, response of radiation detectors, dosimetry and characterization of ionization chambers, radio-therapy and simulation of medical electron accelerators and cobalt units, and synchrotron radiation.

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MC CODES

□ FLUKA <u>http://www.fluka.org/fluka.php</u>

Particle interactions

	Secondary particles	Primary particles
charged hadrons	1 keV – 20 TeV	100 keV – 20 TeV
neutrons	thermal – 20 TeV	thermal – 20 TeV
electrons	1 keV – 1000 TeV	70 keV - 1000 TeV (low-Z material)
		150 keV - 1000 TeV (high-Z material)
photons	1 keV – 1000 TeV	7 keV – 1000 TeV
heavy ions	10 MeV/n – 10000 TeV/n	100 MeV/n – 10000 TeV/n

Applications in high energy experimental physics and engineering, shielding, detector and telescope design, cosmic ray studies, dosimetry, medical physics and radio-biology.

Particle methods for laser-produced plasmas

What was not covered here?

- Particle-in-cell simulation method for macroscopic degenerate plasmas https://doi.org/10.1103/PhysRevE.102.033312
- The Pretty Efficient Parallel Coulomb Solver
 https://www.fz-juelich.de/en/ias/jsc/about-us/structure/simulation-and-data-labs/sdl-plasma-physics/pepc

Thank you for attention

doc. Ing. Ondřej Klimo, Ph.D. FJFI CTU in Prague, ELI ERIC ondrej.klimo@fjfi.cvut.cz

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